wood.

Graphite D&D Options, Risks & Opportunities

Ignalina R3 Reactor D&D Workshop 15th November 2018

woodplc.com





Wood attendees



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Graphite D&D, options, risks and opportunities

- A few facts about Wood
- A few facts about graphite and core degradation
- Wood support to UK Advanced Gas-cooled Reactors
- International collaboration
- Options & risks for core dismantling and graphite disposal:-
 - RBMK specific dismantling and disposal issues
 - International collaboration on graphite and decommissioning
 - Dismantling options and graphite treatment
- Opportunities

A few facts about Wood



delivery across broader market and sector spread



Oil & Gas



Alternative Energy



Mining & Minerals



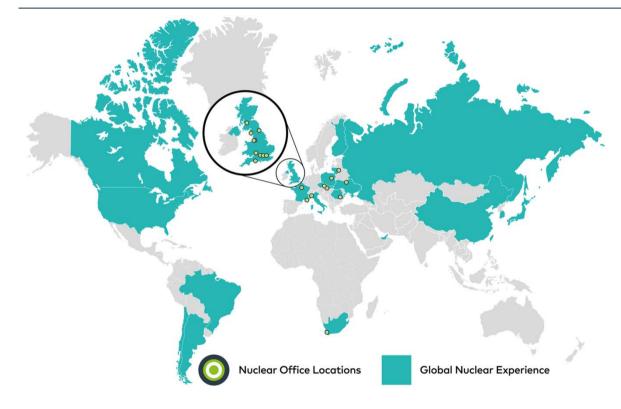
Power & Utilities



Process & Chemicals

4 A presentation by Wood.

A few facts about Wood - our nuclear business



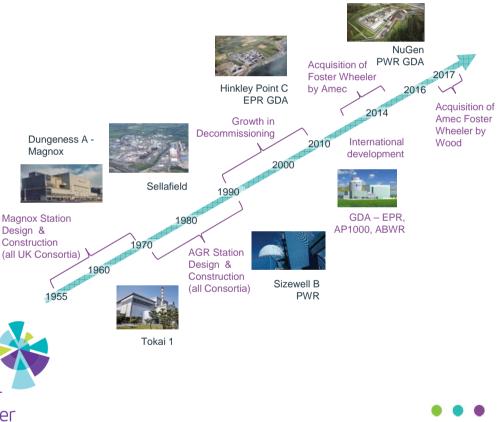
- 2000 people
- 21 regional offices
- Over 20 site offices
- Across UK, Europe and South Africa plus customers in Japan and China.

5 A presentation by Wood.

A few facts about Wood - 60 year nuclear heritage

- The nuclear business has played a key role in the design and build of every civil nuclear power station in the UK; and is playing a significant role on new build programmes, operations and decommissioning across the globe.
- Alongside this heritage, we continue to drive investment in technology and innovation, to help shape the future of nuclear energy.





A few facts about Wood - Graphite Operations & Decommissioning

Operations

Providing customers with:

- Graphite core neutronics and thermal hydraulic modelling
- Graphite component and core modelling/analysis to support operational safety cases.
- Full scale and small scale core rigs to validate computational models.
- Remote handling and in core inspection services.

Decommissioning

- Involved with decommissioning of the Jason reactor at Royal Naval College at Greenwich
- Carried out a feasibility study of the decommissioning options for Tokai Mura in Japan
- Produced major bid for decommissioning the Windscale Piles





Radioactive Waste
Management

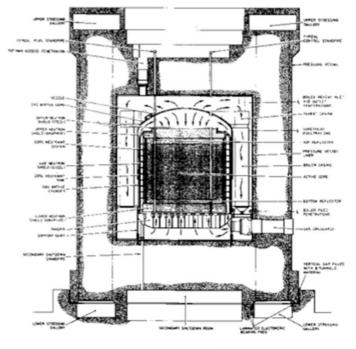


A few facts about Graphite

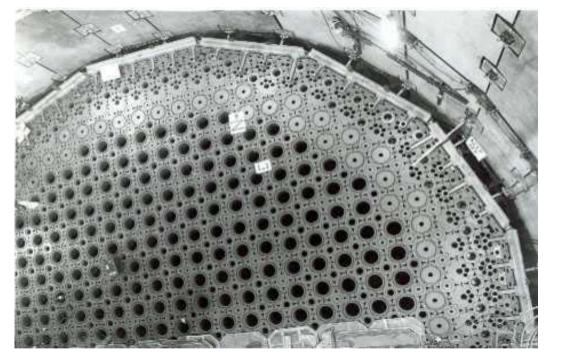
- In excess of 250,000t of irradiated graphite world-wide. 3,400t at Ignalina
- Graphite is 'damaged' by neutron irradiation... graphite components may therefore become distorted, cracked, and so difficult to extract conventionally:
- Wigner Energy could be present if irradiation was at low temperature
- Impurities will have become activated during operation with ¹⁴C produced from ¹³C and ¹⁴N : also ³H from moisture and fuel and, in some cases, contamination from failed fuel fission products
- If cooled by air or carbon dioxide, graphite components/core structure also weakened over life by chemical oxidation (weight loss)
- An issue not only for major historical producers (*e.g.* UK, Russia, France) but also for any country with small reactors containing graphite reflectors or thermal columns



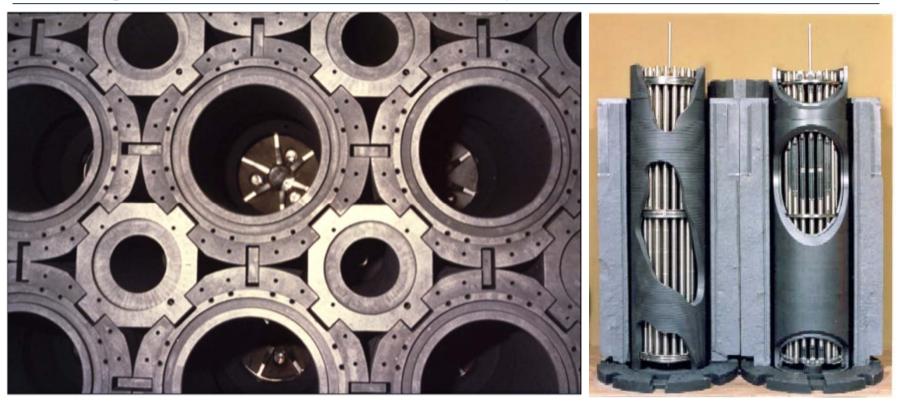
Heysham 2/Torness AGR



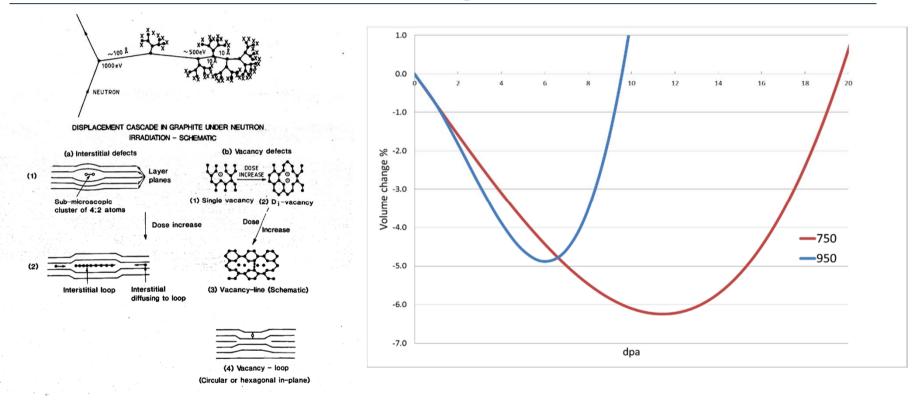
AUDINEMENT OF REACTOR AND PRESSURE VESSEL



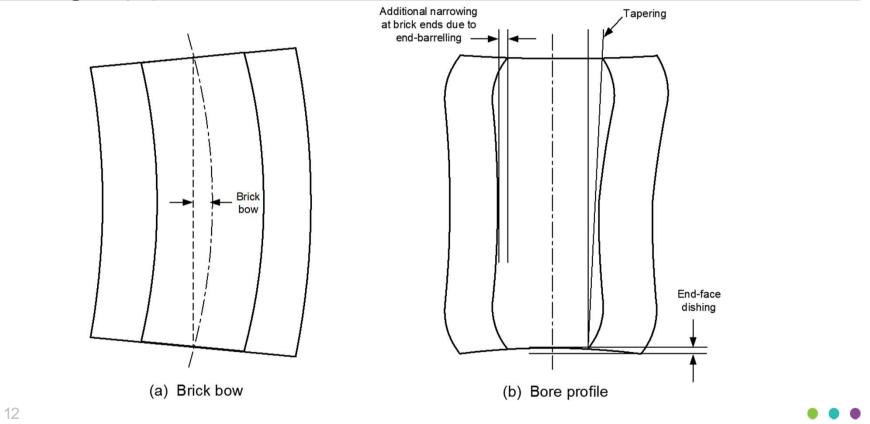
AGR graphite core bricks (Heysham 2/Torness)



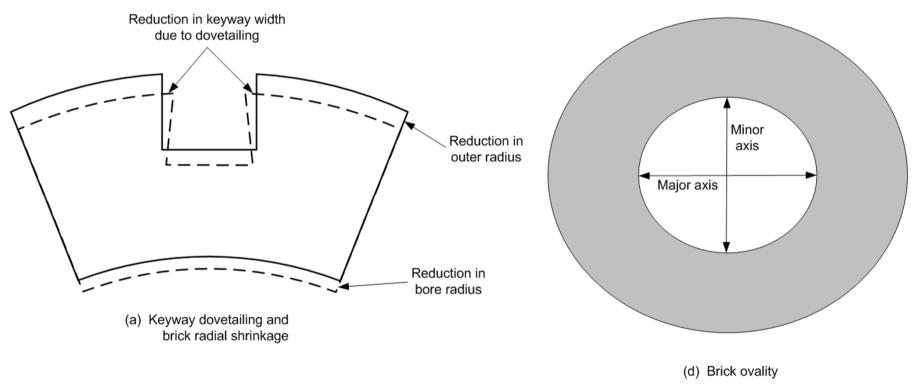
Graphite dimensional change



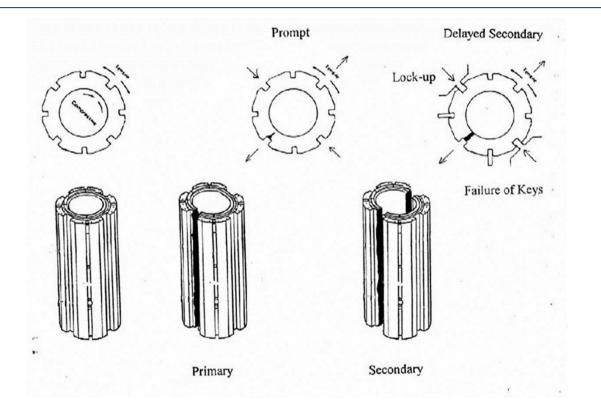
AGR brick distortion due to graphite dimensional change (1)



AGR brick distortion due to graphite dimensional change (2)



AGR brick cracking



Solid model of a brick array



3D full scale rig



2D slice rig (1)



2D slice rig (2)



1/8th scale rig

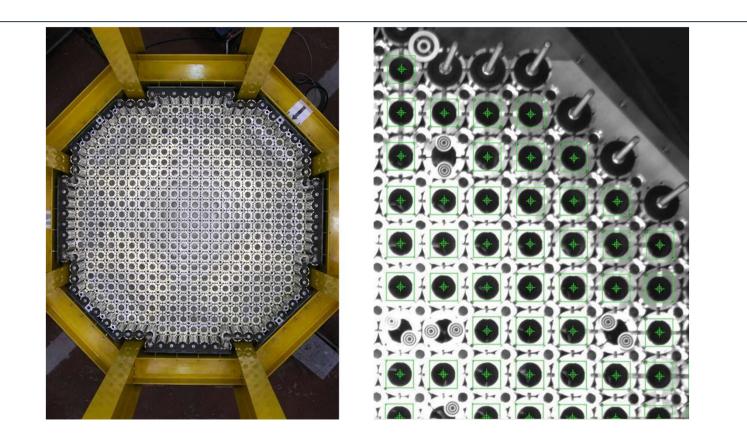


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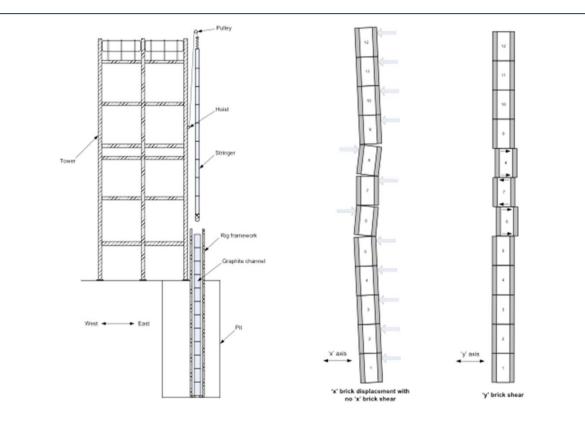
Quarter scale rig (1)



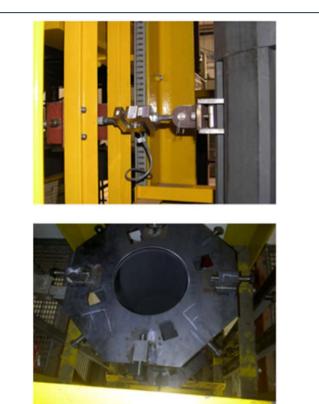
Quarter scale rig (2)



Fuel channel rig (1)



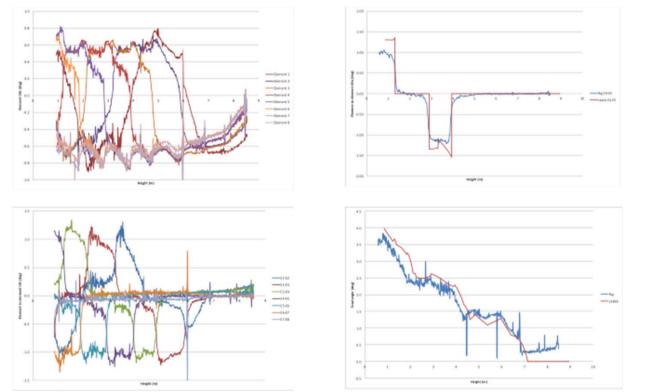
Fuel channel rig (2)







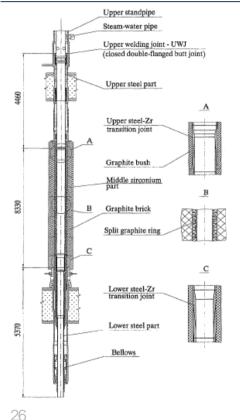
Fuel channel rig – comparison of model predictions and measurements



Core integrity/degradation in RBMKs (1)

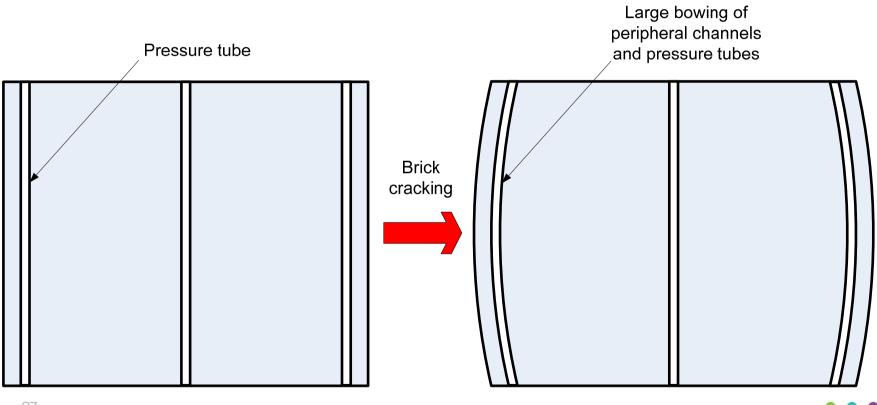
- 'Graphite moderator lifecycle behaviour'. Bath University, 1995. IAEA TECDOC 901 Three papers on RBMKs:
 - 'Radiation damage and life-time evaluation of RBMK graphite stack'
 - 'Assessments of the stresses and deformations in an RBMK graphite moderator brick'
 - 'The state of the graphite stack at Leningrad NPP Unit 2 after 16.5 fpy of operation'
- 'RBMK fuel channel integrity'. IAEA-EBP-RBMK-05, January 1999

Core degradation in RBMKs (2)



- Brick cracking in an RBMK will eventually occur in the same way as for the AGRs.
- Subsequent expansion causes a gap to appear between the rings and the bricks which increases the brick temperature..
- The bricks push on each other and this increases the core diameter.
- This causes bowing of the fuel channels and hence the pressure tubes. The effect increases with core radius.
- The predicted time of operation for brick cracking to start at Ignalina NPP was 22 years

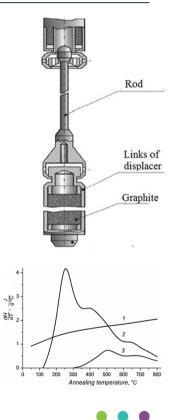
Core degradation in RBMKs (3)



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RBMK specific dismantling and disposal issues

- Possible brick cracking and core instability during/after pressure tube removal leading to difficulty with graphite block removal
- Possible high Wigner Energy levels in the graphite associated with the control rods which will have operated at low temperature
- Possible high ¹⁴C levels formed from the nitrogen in the gas blanket
- There have been, and still are, a number of international collaborations on graphite and decommissioning concerning dismantling options and graphite treatment prior to disposal

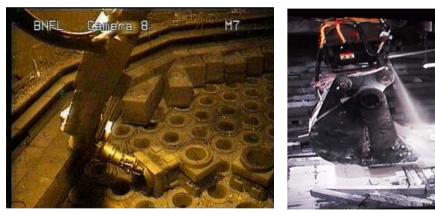


International collaboration on graphite and decommissioning

- CARBOWASTE 'Treatment and Disposal of Irradiated Graphite and other Carbonaceous Waste' April 2008 to March 2013
- GRAPA 'Irradiated **GRA**phite **P**rocessing **A**pproaches' Ongoing
- IAEA Co-ordinated Research Projects TECDOCs
- International Nuclear Graphite Specialists Meetings held annually in September
- GenIV International Forum (Very) High Temperature Reactors and Molten Salt Reactors

Dismantling Options

- Conventional Handling and Cutting
- Under water (reasons which have been advanced include shielding and concern over dust behaviour);
- Special techniques integrated with disposal philosophy



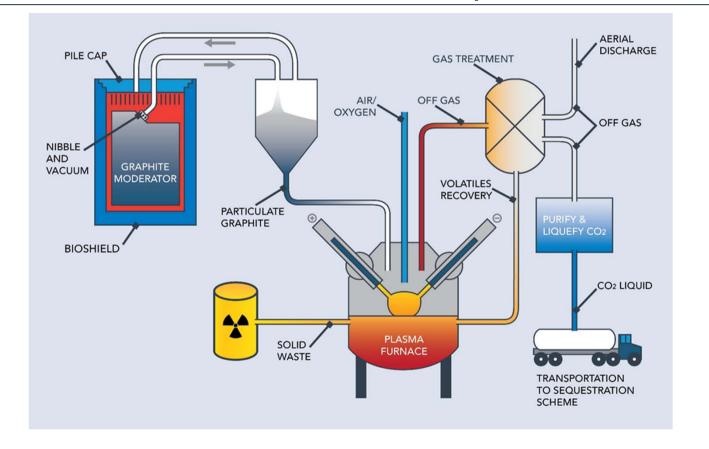
Conventional Handling and Cutting



'Nibble and Vacuum'



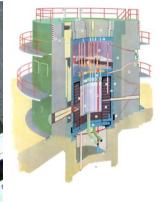
Integrated Dismantle and Treat Option



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Treatment Options - Immobilisation





Cementation Paul Scherrer Institute Switzerland DIORIT (research reactor) 45 t Graphite treated



Vitrification in Molten Salt

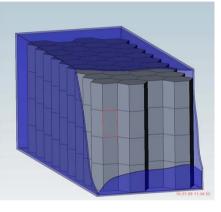


Treatment Options - Immobilisation

Graphite-Loaded Glass



High density (2.23 g.cm⁻³) – regarded as 'impermeable' and non-corrodible



IGM density ~ 2.2 t/m³ graphite density within the block ~ 1.6– 1.7 t/m³ graphite package density in container ~ 1.5 t/m³ 80,000t graphite ~50,000m³

80% graphite currently, used to Immobilise Other Radwaste (FNAG, Germany)



Treatment Options - Incineration



Fluidised-Bed Incineration



Treatment Options - Reduce Radioactivity Content (Leaching and Washing)

- Leaching rates in a repository or storage environment need to be extremely low: data in 'alkaline liquors' are most relevant but timescale of tests <<<< repository life (geological timescales)
- ...<u>but</u>: valuable to explore 'washing' options in other aqueous environments in order to facilitate isotope removal/recovery ahead of disposal

Treatment Options – Heat treatment

- Most of the induced activity of the graphite will have decayed away within ~70 years after plant closure. However, the remaining ¹⁴C concentration, which will be potentially much higher in the RMBK graphite than other reactor graphites due to the activation of the ¹⁴N in the gas blanket, means that it is classified as intermediate level waste.
- Removing a significant amount of the ¹⁴C content may allow it to be reclassified as low level waste which could lead to large reduction in disposal costs
- Work undertaken under Carbowaste showed that heat treatment was one of the best options. The UK is investigating this form of treatment further with INNOVATE UK funding

Opportunities

- Storage or Recycling ?
- Future market for nuclear graphite ?
- High cost to store and to dispose, so if it is industrially feasible it makes sense to recycle. Options with real graphite waste currently under investigation.
- Two major graphite manufacturers participating in the EU CARBOWASTE programme established that the process is feasible but requires significant investment in a production plant to handle moderately radioactive feedstocks.



